

The International Thorium Molten-Salt Reactor Developmental activities

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**International Thorium Energy & Molten-Salt Technology Inc.*

(IThEMS, Chairman)

***International Thorium Molten-Salt Forum (ITHMSF, President)*

October, 2010

- ❑ Global Major Technology : 100-200 years life
- ❑ Based on the **SYSTEM – PRINCIPLE**
SYSTEM is “**nuclear fuel-cycle system**”
(not “nuclear power reactor”)
- ❑ “**Simplicity**” ----- reliable ----- “ECONOMY”
- ❑ **Flexible System:**
better is better (best is not best)
----- familiar technology

1944: Chicago University (Eugene WIGNER)

Reactor : chem. Engineering system

working medium "LIQUID"

(might be "molten fluoride salts")

1950- 1980: ORNL (Alvin WEINBERG, MacPHERSON,)

Molten-Salt Reactor PROGRAM

1950- 1976: CRNL (W.B. LEWIS)

Accelerator Breeding : SPALLATOR

1980: JAERI (K. FURUKAWA, K. TSYKADA, Y. NAKAHARA)

Accelerator Molten-salt Breeder (AMSB)

1985: FUJI (fuel self-sustaining simple MSR station)

1988—: THORIMS-NES

(**Thorium Molten-Salt Nuclear Energy Synergetic System**)

“Brief HISTORY of Molten-Salt Reactor DEVELOPMENT”

- 1947—76: Oak Ridge NL: MSR-Program [MS Breeder Reac.(MSBR) development]
- 1965-69: Experimental-Reactor: MSRE operation (total :26,076 hrs, no accident) at ORNL
- 1968-80: USA: Molten-Salt Group (EBASCO, etc.: Reicle, deBoisblanc)
approached to JAPAN for MSBR develop. (Kamei, Nishibori)
- 1972-83: EdF [Elec. de France]-CEA MSBR program (Bienvenu, Lecocq)
approached to KF from 1981
- 1980.10 KF etc. invented AMSB (Accelerator Molten-Salt Breeder).
- 1981 Academic Assoc.”Thorium-Energy Research”(Prof. Kaya, Husimi, Nishibori, Saito etc)
Parliament Group for Th Energy in Govern. Political.Party(100members)
- 1982.5. The 1st MSR Meeting at EUCHEM-Conf., La Gaillarde, France
- 1982 USSR Journal "Atomic Energy in Abroad" translated the KF paper of AMSB.
- 1983.6 USSR-Kurchatov Inst. (KI) (Direc. Acad. Alexandrov, Dr. Novikov) proposed MSR-
Cooperation Development to KF.
- 1985.8 KF etc. : invented the fuel-self-sustainable simple small MSR : FUJI

- 1986.3 USSR-KI decided **MSR Construc.** (leader: Dr.Legasov) just before Chernobyl-accident [1988.4: Dr. Legasov died & stopped MSR project]
- 1987.11 EdF decided “no SUPERPHENIX-No.2 (Na-FBR)” (President Bergougnoux) and invited KF for the Th-MSR examination
- 1988.11 ORNL(Direc.Trivelpiece) proposed to KF the coop. on MSR with USSR..
- 1988.12 KF prepared **“THORIMS-NES Report”** at Clamart Inst., **EdF**,
- 1990.5 USSR-KI (Direc.Velikhov) proposed to KF the coop. on MSR devel. again.
- 1991.7 USSR-Inst.Theo.Exp.Phys.(Chuvilo) invit.KF coop. AMSB-study, & Sosny Sci. Cent.(Chigrinov), Minsk, too.
- 1992.6 US Presi. Bush’s Advisor.Sci. &Tech.(Dr.Allan Bromley) encour.KF on Th-MSR devel.
- 1995.6 RUSS Fed. Inst.Tech. Phys. (ITP), Snezhinsk. (Direc. Acade.E. Avrorin, Dep. Sci. Direc, V, **Simonenko**), proposed the **coop. devel. of miniFUJI. and accepted by KF**
- 1997.4 **“Intl Conf. MSR Develop.”**, at RAND, USA with 24 MSR specialists. from **Japan(5), USA(5), Russia(4), Belarus(2), India, France, Turkey, Czech & IAEA**
- 1997.7 **Japan·USA·Russia Trilat.Coop.MSR-Dev Meet.**, decid.miniFUJI-site in ITP, Snezhinsk.
- 1997.8 US Presi. Clinton Advisor Sci.&Tch.(Dr.J.Gibbons) promised to KF, Japan,USA,Russia Trilat. Coop.MSR-Dev. and Coop. with ORNL: no problem

- 1998.2 Academ.Avrarin, ITP visit Japan; Russian Govern. approved MSR Coop. plan.
- 1999.12 ***Three Agency Study (OECD/IEA, /NEA, IAEA) [TAS]***: Intl. Devel. of 12 Nuc Pow. St., incl. FUJI.
- 2001.8 pub. Book:“**Revolution in Nuclear Power-Plants**” (Japanese) by KF. in success
- 2002.10. **[TAS]** Report pub.:”Innovative Nucl.Reac.Devel-Opportu. Int.Coop.”
- 2003.10 Parliam.Group of Govern.Party invited KF on Th-energy policy.
- 2004.5. IAEA “Status-Rep. Innov. Small & Medium Sized Reactor Design”
(2004.10. our 3rd draft was sent to IAEA), will be pub.Oct.,2006
- 2004.9 Czech & Slovakia Visit: , Rez NRI; Pilzen SKODA; Brno; Slovakia Nucl. Ene. Conf..
- 2005.8 Compreh.Rep. ”New Primary Energy by Th MSR Tech.” Elec.Chem.73 (2005) 552.
- 2005.9 R.Moir & Edward Teller: Nucl.Tech.151 (2005) 334. supporting FUJI concept.
- 2006.2 GIF-MSR Steer.Comm.Meeting, at OECD, Paris (observer: KF)
- 2006.6 “**Nucl.Non-Prolif.**” by KF awarded Prize of Excellence from Nobel Peace Prize Winner Eisaku Sato (the former Japan. prime minister) Foundation.”

- 2006.9 **SKODA-JS**(Pres. M.Fiala) invited KF to the 50 years anniversary
- 2007.1. **IAEA-TECDOC-1536**: “Status on Small Reactors without On-site Refuelling” published including “FUJI concept”.(p.821-856)
- 2007.8. **ICENES, Istanbul, Turkey**: “ Road Map Realiz. Global-scale Th Breed. Fuel Cycle by Single Molt.-Fluoride Flow”, 18 coauth. [Ene. Conv. & Manag., 49(2008)p.1832-1845.]
- 2008.6. 21st Int.Conf.ECOS 2008, Poland: present. 2 papers by Prof.Erbay(Turkey) et al.
- 2008.10. Joint Int. Conf. on MS:MS08, Kobe, Japan: presented.
- 2008.10. **Formal NPO: ITHMSF** (Int. Thorium Molten-Salt Forum) was established.
- 2009.4. Taiwan Inst. Nuclear Eng. Research (INER) engineers visited Furukawa
- 2009.4. Adv. Nucl.Fuel Manag.(ANFM IV),Th-fuel Sec. South Carolina,USA: present 2 papers
- 2009.4. visited Venezuela, and discuss. with USA colleagues at Los Angeles.
- 2009.5. discuss with Dr. A. Bjorseth, Chairman of SCATEC, mother comp. of Thor Energy.
- 2009.6 visit Prague and Moscow for miniFUJI & FUJI development task plan
- 2010.3 present. at “Th Energy Alliance” Conf., March 29-30, 2010, Mountain View, CA, USA
- 2010.6 started the new company: **International Thorium Energy & Molten-Salt Technology Inc. (IThEMS)**: President K.Fukushima, Chairman K. Furukawa
- 2010.9 visiting Czech: IThEMS and Czech side signed memorandum for collaboration towards the realization of a miniFUJI in practical use in a near future.

“Revolutionary Strategy for Our Target : THORIMS-NES (Thorium Molten-Salt Nuclear Energy Synergetic System)”



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October, 2010*

Agenda

☒ A. Basic Principle of Fission Energy Industry Technologies:

“Nuclear Fuel-Cycle” is much more important than “Reactors”

☒ B. Selection of specific Fuel-Cycle System:

☒ C. Path to realize the Th-U Breeding Fuel-Cycle:

☒ D. Development strategy for realizing the “Simplest”

Th-U Breeding Fuel-Cycle:

☒ Conclusions:

A. Basic Principle of Fission Energy Industry Technologies:

(1) “Establishment of Breeding Nuclear-Fuel Cycle” is crucial:

- ❑ The **doubling time (Td)** for fission fuel industry growth **in about 10 years** (cf. the next-page Figure) for establishing the requested fission fuel breeding cycle.

-Necessary for Global Survival-

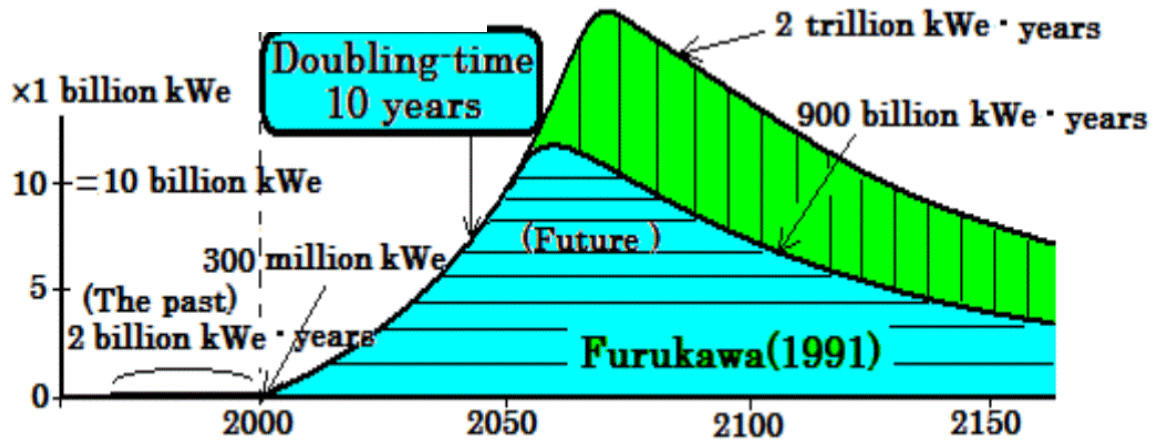
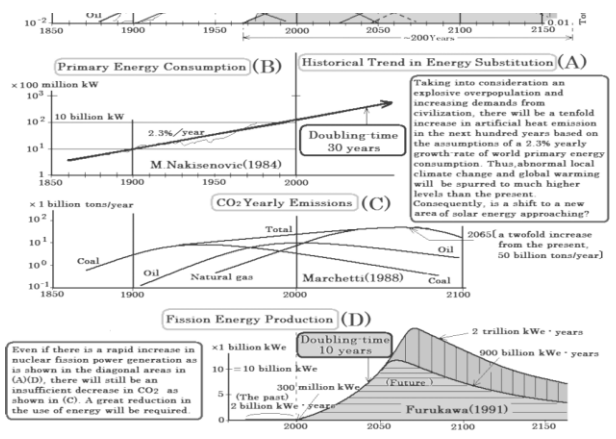
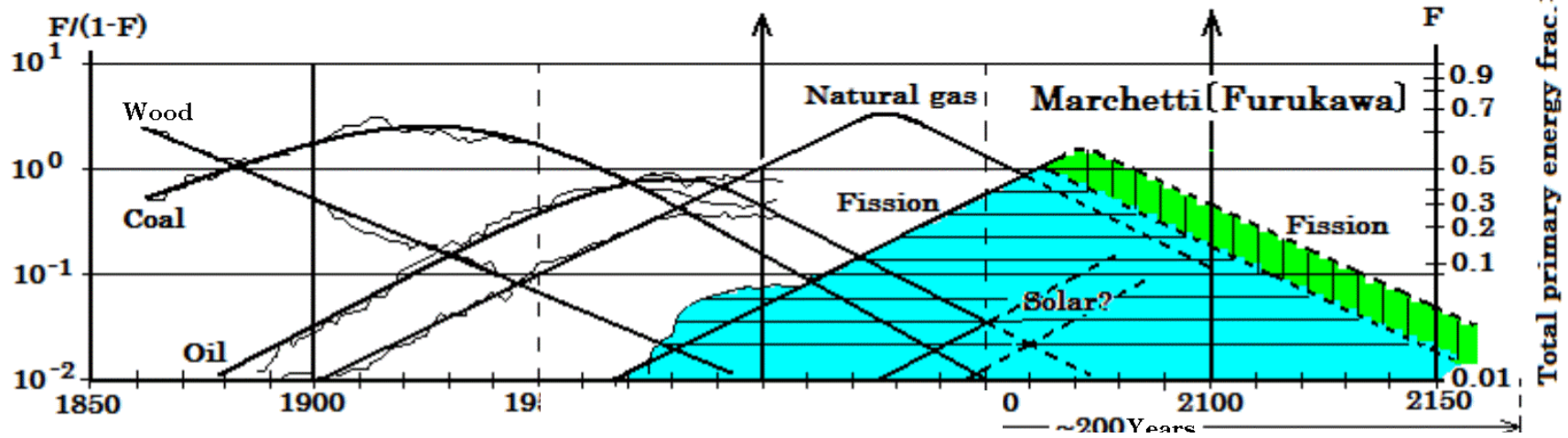
- ❑ Only require 10,000 tons of the consuming fission fuel per year for 10-TWe (10 billion kilo) , with its establishment:
→ **Nuclear waste can be drastically reduced.**

“Global Future Energy Prediction”

Logistic Function

The basic problem: disordering of major energy technology, Application of solar-based technology: Needs 100 years more

$$\frac{\text{artificial heat emission}}{\text{solar heat input to earth}} = \frac{1}{10,000} \xrightarrow{(2.3\%/\text{year}) \text{ 10 times}} \frac{1}{1,000} \text{ (abnormal micro climate)}$$



A. Basic Principle of Fission Energy Industry Technologies:

(2) Breeder Power Reactor system is too “utopian”:

- ❑ Breeding Cycle should be achieved by a **Symbiotic system**:
- ❑ FISSION process is: **Energy rich**, but **Neutron poor**
“Fission Breeding Power Reactor” is insufficient
- ❑ SPALLATION process: **Energy poor**, but **Neutron rich**
- ❑ A symbiotic system couples:
 - Good converter of Fission Power Reactors with
 - Good fissile producing Spallation Reactors
- ❑ **“Near Breeder”** (“Self-sufficient nuclear fuel system”):
 - Most simple, most stable and very high safety–
 - ➔ the **ideal NUCLEAR POWER STATION !**

B. Selection of specific Fuel-Cycle System:

(1) Uranium – Plutonium (U–Pu) breeding fuel-cycle is the subsidiary technology of early stage of the development:

- ❌ **Disadvantages** of SOLID-FUELed U–Pu Breeding Fuel–Cycle
 - nuclear proliferation --- **plutonium**
 - nuclear waste ---- **trans-U elements**
 - possibility of severe accident (core melt-down) --- **weak safety**
 - economical difficulties due to:
 - “fuel-assembly fabrication & handling”,
 - “complex reprocessing”, “necessity of huge-size”, etc.

- ❌ Huge investment in the past 60 years, but not successful in realizing an effective Breeding Fuel–Cycle:

- ❌ But, its contribution will remain until the middle of this century.
 - ➔ **The accumulated technology** has made launching the new technology easy and feasible:

C. Path to realize the Th-U Breeding Fuel-Cycle:

(1) Establishment of the Nuclear-Fuel Cycle (NFC) as "Chemical Engineering Devices" (1/2):

- ✖ What Dr. Eugene WIGNER predicted in 1944 was that:
 - **REACTOR:** Nuclear Chemical-Reaction Energy System
 should be **"Chemical Engineering Devices"**
 - Its working medium should use a **"FLUID"** as the nuclear fuel
 and as coolant
 - An **ideal** nuclear power reactor would be probably
"the molten-fluoride salt fuel reactor"
 later developed by ORNL, USA under his successor:
Dr. Alvin Weinberg

B. Selection of specific Fuel-Cycle System:

(2) The Thorium - Uranium (Th-U) breeding fuel-cycle is the most promising approach:

- ❑ Superiority of the “Th-U” over the “U-Pu” system is extensively recognized in these days:
- ❑ One of the superiorities:
Almost **none of “the trans-U elements”** is produced:
- ❑ **Weak point:**
No “natural fissile nuclide” available with this “Th-U”:
→ Alternative solution available:
- ❑ The “Th-U” system should be the mainstream of the development strategy:

C. Path to realize the Th-U Breeding Fuel-Cycle:

(1) Establishment of the Nuclear-Fuel Cycle (NFC) as "Chemical Engineering Devices" (2/2):

❑ Thorium utilization in "SOLID-FUEL REACTORS":

High Temperature Gas-cooled Reactors (HTGR) and Light- or Heavy-Water Reactors (LWR or HWR) etc.

- Not suitable for practical "Th-U" cycle development due to the **strong gamma rays associated with U232**:
- Difficulties of **Chem.-Processing and Fuel-Fabrication**



C. Path to realize the Th-U Breeding Fuel-Cycle:

(2) Fissionable material procurement:

- ❑ Thorium is a **fertile material** and has no fissile nuclide such as U235 or Pu239/241:
- ❑ Slightly enriched or **denatured uranium** can be used for the **initial fuel** (Plutonium produced will be of relatively low concentration and small in amount) :
- ❑ **Plutonium** from spent uranium solid-fuels, resulting by **simplified FREGAT processing** (no fuel-assembly and fabrication)
- ❑ Eventual elimination of the plutonium stockpiles:

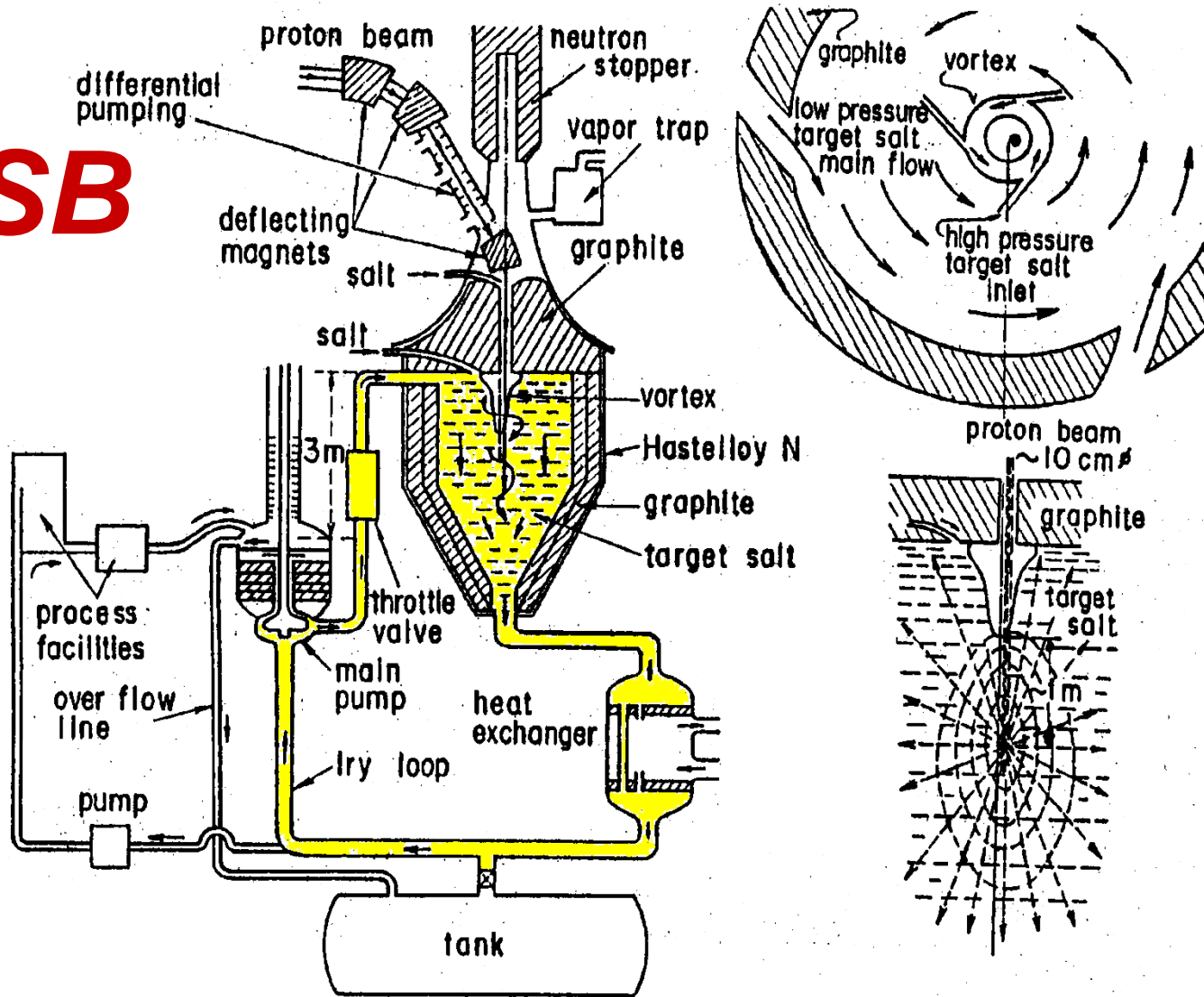
C. Path to realize the Th-U Breeding Fuel-Cycle:

(3) Realization of Breeding Fuel-Cycle by selecting the Symbiotic system:

- 
 The key factors for establishing the full scale breeding fuel-cycle:
 - 
Implementing the comprehensive improvement and maintenance of the
 - “Accelerator Breeding System”**
(SPALLATOR)

- 
 Successful development of the
 - “Accelerator Molten-Salt Breeder (AMSB)”**
 - invented by K.F. et al. on 1980 (cf. the next-page Figure)
 - 
Solving the technical difficulties of:
 - intensive radiation-damage &
 - localized intense heat generation etc.:

AMSB



D. Development strategy for realizing the “Simplest” Th-U Breeding Fuel-Cycle:

(1) Basic principle of new technology:

- ✖ **CANADIAN PLAN** in realizing the “Th-U Breeding Fuel-Cycle”:
 - Solid-fuel high conversion-ratio “**CANDU**” + “**Accel. Breeder**” was considered by Dr. W. B. Lewis (1950–1980)
 - Not realistic to achieve a simple breeding cycle system.
Recent Canadian plan: “**CANDU + FBR**” is much worse
 - **INDIA** is in a similar situation at the moment

- ✖ **THORIMS-NES concept** based on Breeding Fuel Cycle by the “**Single Molten-Salt Liquid Phase**” to be implemented:
 - (MSR: **FUJH**) + (Accelerator Molten-Salt Breeder: **AMSB**)

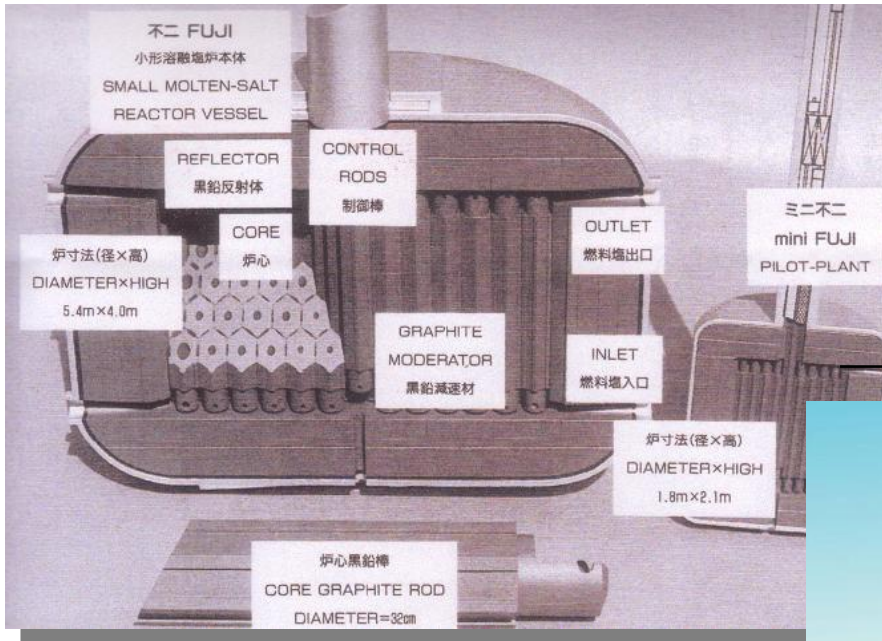
D. Development strategy for realizing the “Simplest” Th-U Breeding Fuel-Cycle:

(2) Idealistic “Thorium – Molten Salt Reactor” system (1/2):

- ❏ Proposed **Thorium–Molten Salt Reactor**,
“FUJI”: *near breeder* (cf. the next–page Figure)
- **Simplified Structure**, easy to operate and maintain:
 - Almost *fuel self–sustaining*,
 - Even small power size: 150–300 MWe
 without continued chemical processing & core–graphite replacement
- **Simple Reactor Vessel**: Simple ambient pressure tank
 with no big–flange, only one control–rod and no fuel–handling:
- **Factory manufactured**: Small size & modular arrangement:

Excellent economical advantages (Continued)

Molten-Salt Power Reactor

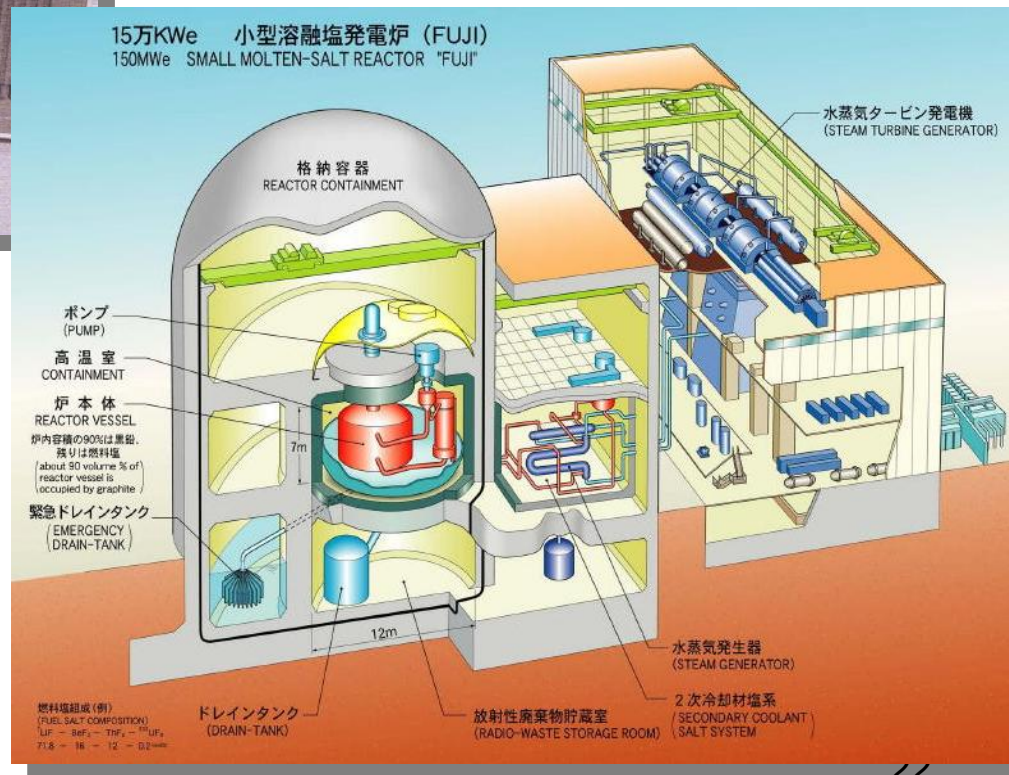


FUJI

Cross-section View of FUJI and miniFUJI Reactor Vessel Models

Inside: **Graphite** is only **90 % vol.**
Almost **no Control-rod.**

Full View of FUJI Molten-Salt Reactor

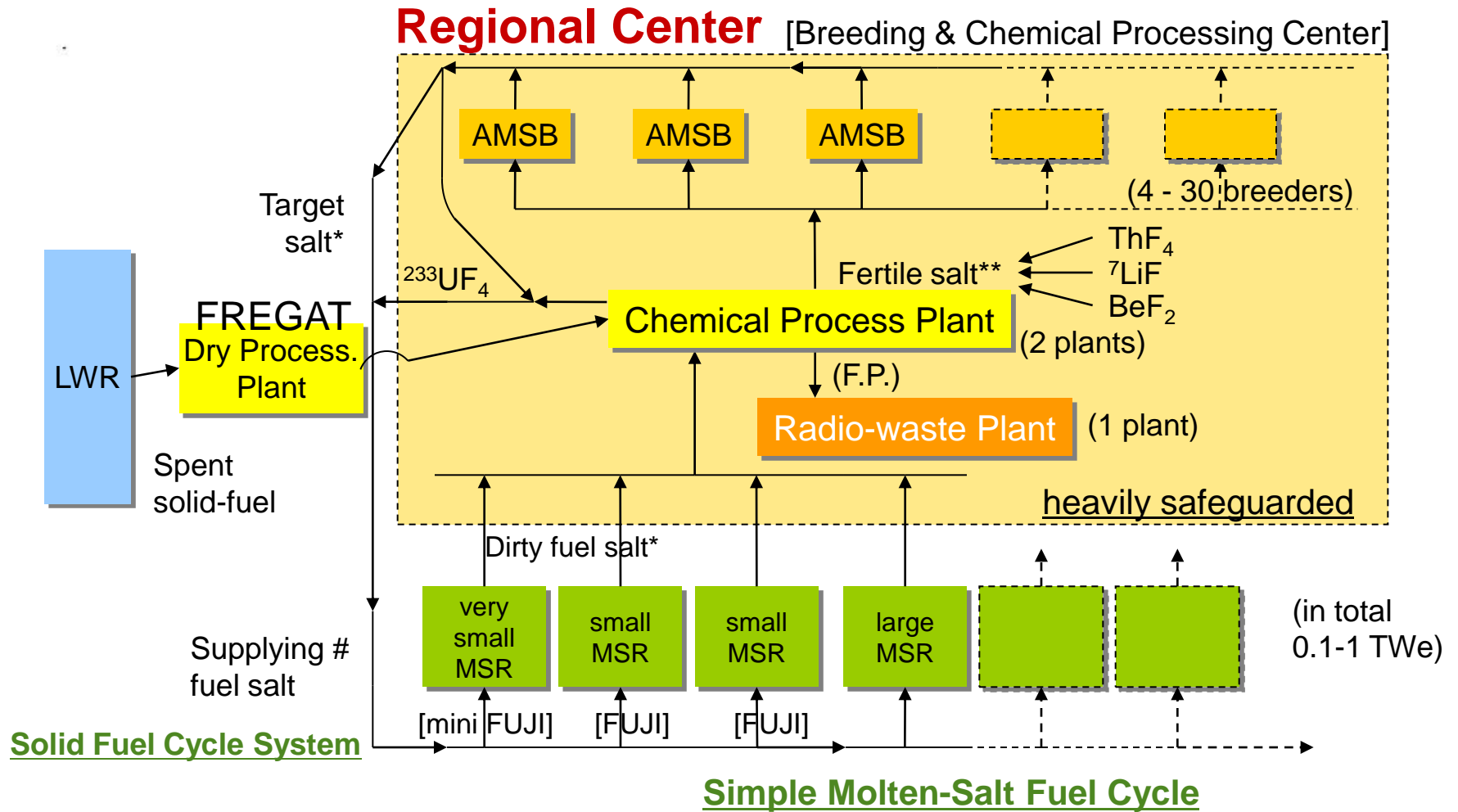


D. Development strategy for realizing the “Simplest” Th-U Breeding Fuel-Cycle:

(2) Idealistic “Thorium – Molten Salt Reactor” system (2/2):

- ❏ For the 1st stage operation fuel:
 - Plutonium** from existing warheads stockpiles and Solid-U-fuel reactors aiming at the reduction and eventual, total, elimination of plutonium
- ❏ For the 2nd stage operation fuel:
 - Production of **U233** for many new reactors: **FUJI** by launching many “**AMSB**” Timetable: about 3 decades
- ❏ “**Regional Centers**” : 20–30 locations in the world (cf. next Fig.) to handle the “chemical processing” of the spent fuel salts:
 - *“U233 producing AMSB” and “Nuclear Waste Plants”*:

Thorium Molten-Salt Breeding Fuel-Cycle System



(*) ⁷LiF-BeF₂-ThF₄-²³³UF₄

(**) ⁷LiF-BeF₂-ThF₄

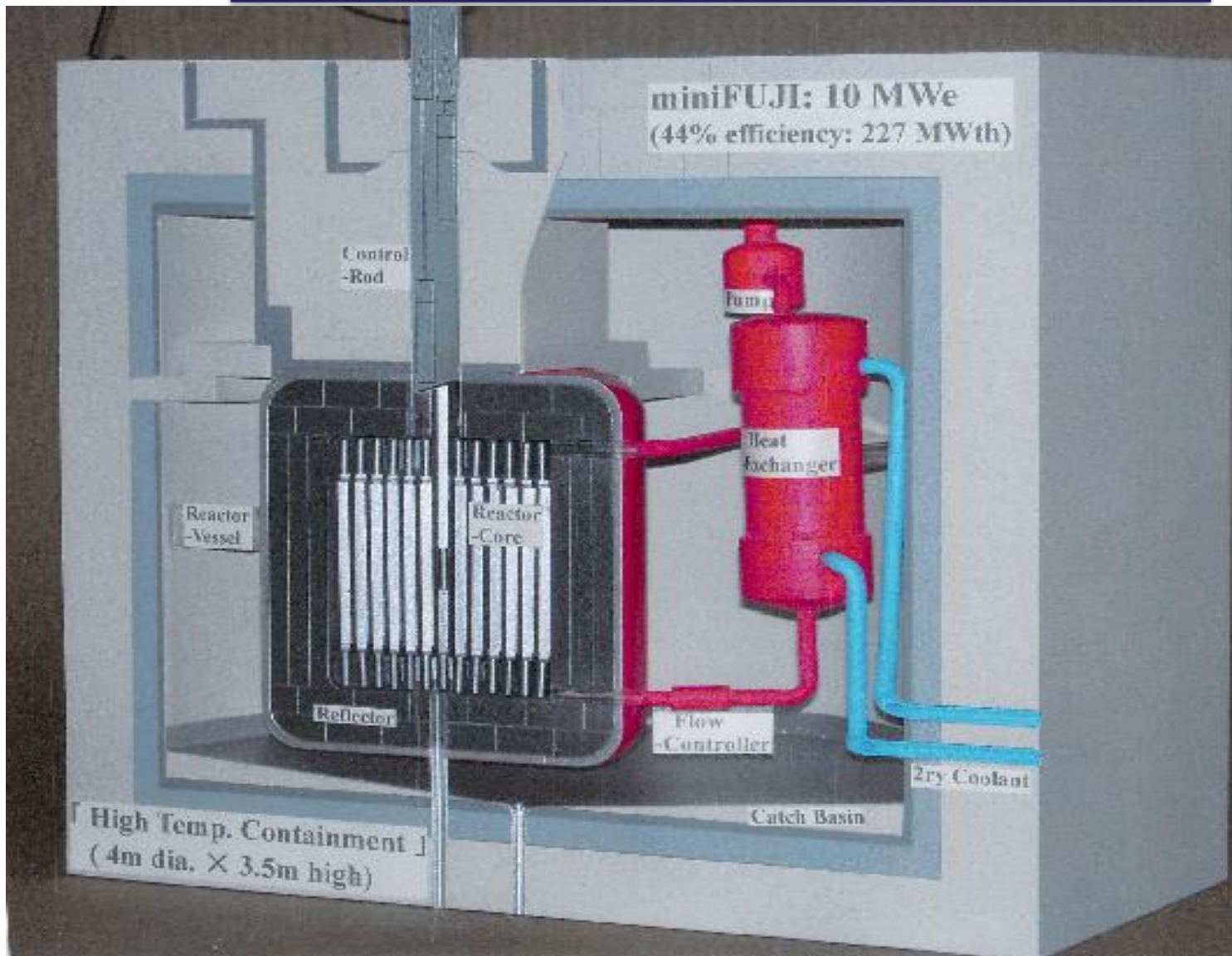
(#) target salt* + additive ²³³UF₄






D. Development strategy for realizing the “Simplest” Th-U Breeding Fuel-Cycle:


(3) THORIMS-NES is feasible in the Shortest Period with Minimum Development Cost:


- ❑ Concept based on the review of *R&D results for past 70 years*:
- ❑ “Small-sized nuclear power station: *in minimum investment*”
 - by using the past excellent R&D results
 - by international cooperative partnership with private companies”
 - by the support of liquid-sodium technology experience
- ❑ *MSRE operation* at ORNL for 4 years:
 - equivalent to the fuel-burning for 10 years at “FUJI” already
- ❑ Investment estimated:
 - \$0.3B for “**miniFUJI**” reactor for the next 5–6 years:
 - \$1.5B for “**FUJI**” reactor for the next 12 years from now:
 - \$20B for **AMSB** development at “**Regional Centers**” worldwide for 25 years (starting 10 years later) from now


Model of miniFUJI in “High Temperature Containment”



- 
Excellent advantages of THORIMS–NES:
 -  **Safety** (No “core meltdown” accident in principle)
 -  **Radioactive–waste** no production of trans–U elements
 -  **Nuclear proliferation** protection: no Pu
 -  **Economy** (simple structure and operation, etc.)
which should result in a conclusive public acceptance

- 
“The simpler is the better.”

- 
 Promising potentialities for launching by (small–fund & short–time)
 the **huge–sized new nuclear industry** throughout the world.

- 
 Significant contribution to the realization of
“Co–existence & Co–prosperity Society” on earth

“Thorium Molten-Salt Nuclear Energy Synergetic System-THORIMS-NES” Concept of realizing “Co-existence & Co-prosperity Society” on earth

Promising potentiality for launching the **huge-sized New Nuclear Industry** throughout the world.

“THORIMS-NES”

No “Plutonium” production

No “Core meltdown”

Small-sized Nuclear Reactor

*Safety, *Radioactive-waste,
*Nuclear proliferation,
*Economy

“Liquid” fuel

LWR, HTGR etc.

“Solid” fuel

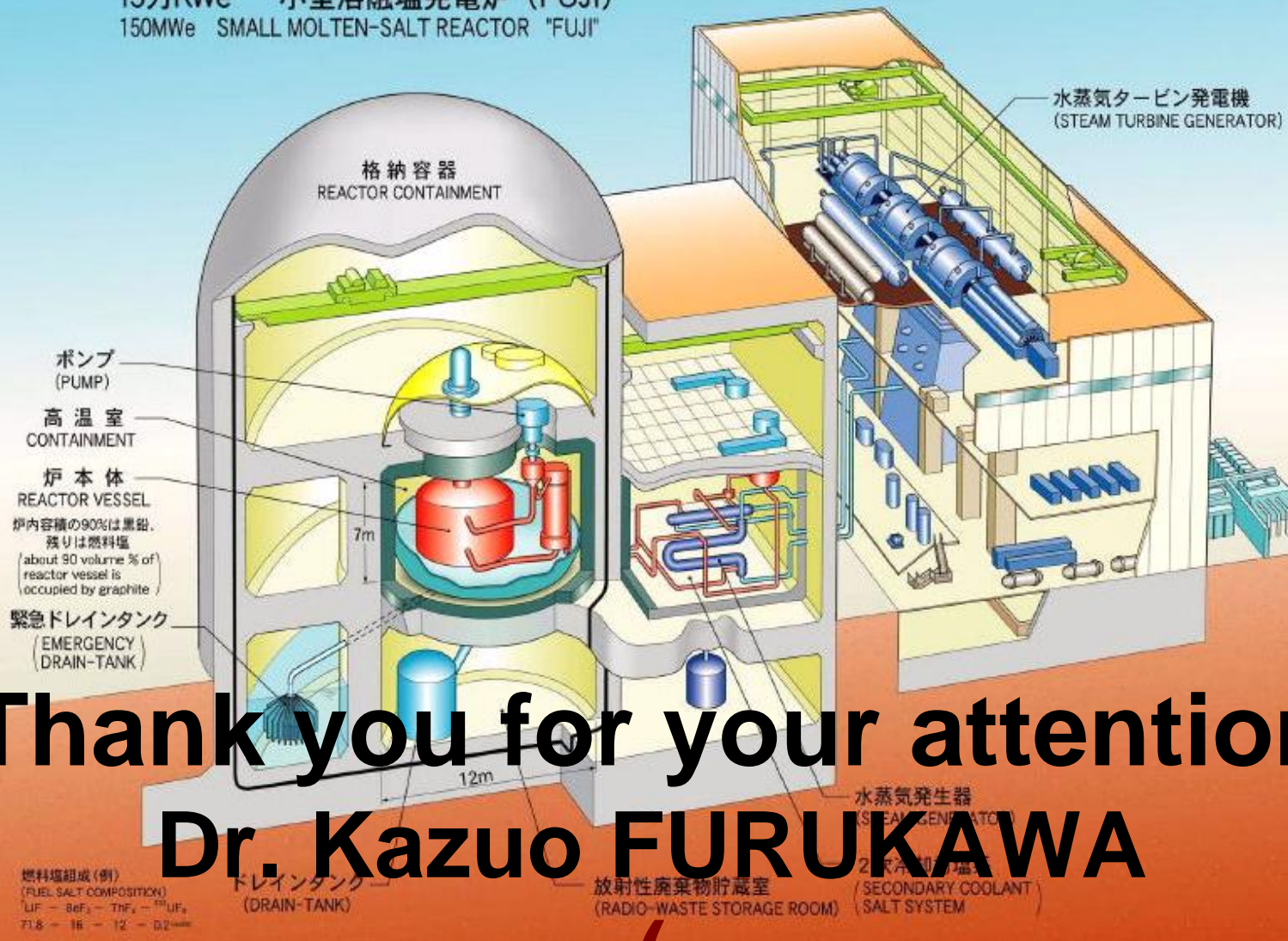
“Plutonium” production

“Core Meltdown”
“Re-criticality” possibility

Huge-sized Nuclear Reactor

- ❑ “Establishment of **Breeding Nuclear-Fuel Cycle**” is crucial:
- ❑ “Breeding Power Reactor system” is too “utopian” and not effective:
- ❑ “U-Pu” Breeding Fuel-Cycle is the subsidiary technology for early stages of the development:
- ❑ Realization of the most promising **symbiotic “Th-U” Breeding Fuel-Cycle**:

15万KWe 小型熔融塩発電炉 (FUJI)
150MWe SMALL MOLTEN-SALT REACTOR "FUJI"



Thank you for your attention
Dr. Kazuo FURUKAWA